

Activities of plant thermal-hydraulics unit

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1. Activities of *new*cleo plant thermal-hydraulics unit

The unit supports the **design and licensing** of *new*cleo LFRs and is responsible for the **thermal hydraulic analyses** aimed at providing deterministic demonstration of the safety of the plant.

The unit will provide **input data** for Mechanical Design, I&C, auxiliary system, etc.

- Analyses by means of thermal-hydraulics codes:
 - System thermal-hydraulics code (possibly coupled to neutron kinetics codes, CFD, etc.)
 - Subchannel codes
 - CFD codes
 - Containment thermal hydraulics codes
 - Severe accident codes, etc.
- V&V of the different adopted codes to be performed



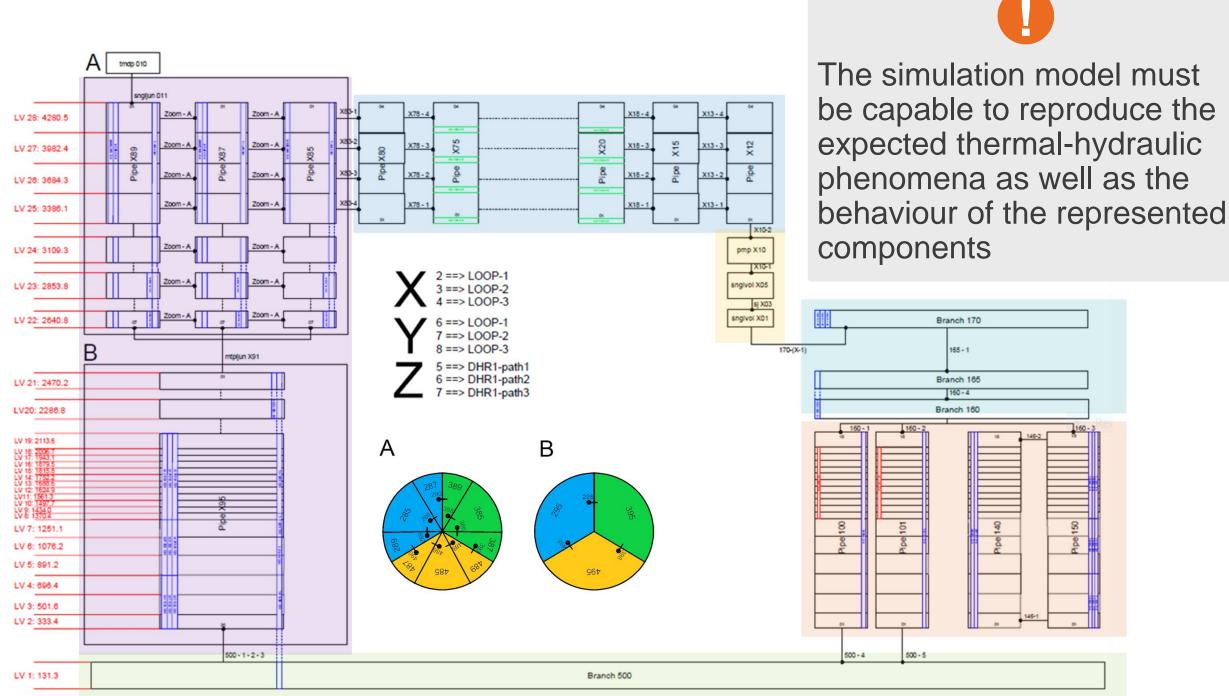
SYS-TH studies





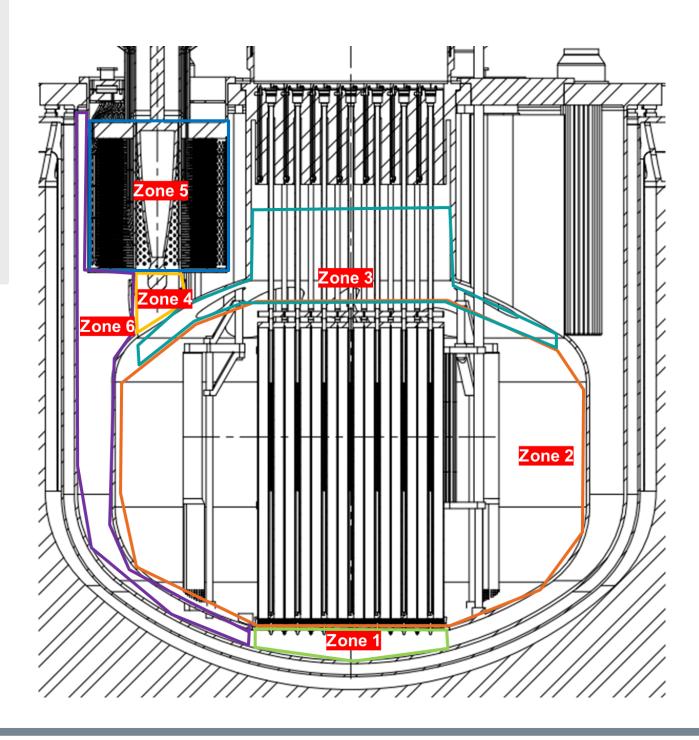
2.a System thermal-hydraulic analyses

- Evaluate relevant parameters to support the reactor design and
- Investigate reactor behaviour in normal, upset and accident situations









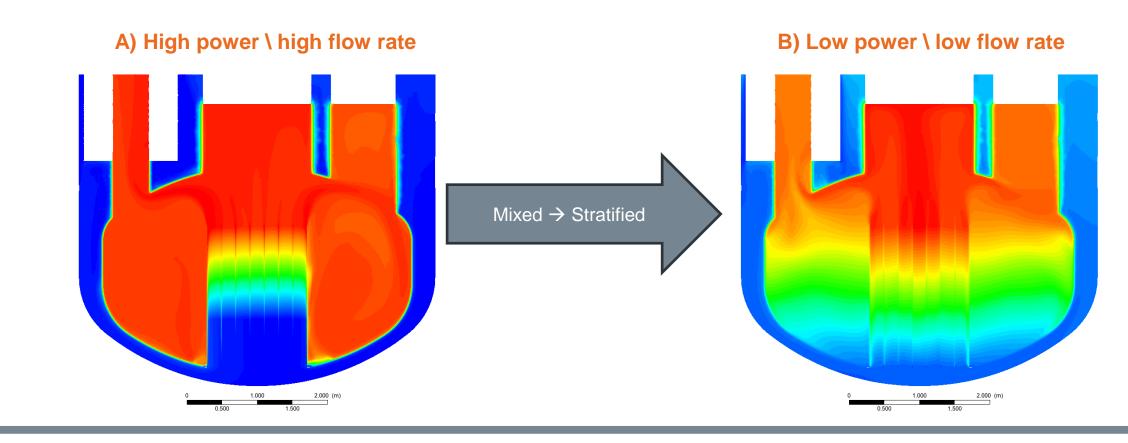
System thermal-hydraulic analyses 2.b

Example of modelling issue:

IN-ASIV pool \rightarrow very complex velocity field to be addressed. Impact on temperature distribution and on the heat transfer with the surrounding downcomer, affecting the natural circulation.

Issues to be considered:

- Effect of pump suction in promoting thermal mixing at different flow rate/power conditions;
- Occurrence or transition to thermal stratification in some operating conditions;
- Convective heat transfer inside the pool.







A) High power \ I B) Low power \ I high flow rate low flow rate 560 540 520 ^{ပ္} 500 Stratified → Stratified 480 460 440 420 400 0 20000 40000 60000 80000 100000 Time [s] 100-01: Average core chan inlet 100-18: Average core chan outlet 150-01: ASIV pool bottom ····· 150-18: ASIV pool top 140-01: IW bottom 140-18: IW top

System thermal-hydraulic analyses 2.C

SYS-TH simulations have been utilized for various purposes:

1. **Thermal Loading Transients:** These simulations support the thermomechanical team in determining the dimensions and loads on reactor components. This is particularly crucial when significant gradients or multiple cycles are experienced.

2. Other Design Requirements: RSSIM simulations support component designers and control system engineers in sizing and designing components. An example is the definition of pump inertia.

3. **Safety Transients:** These simulations are employed to ensure that the reactor complies with safety and regulatory requirements. The objective is to demonstrate the favourable intrinsic safety behaviour of LFR-AS-30 in challenging accidental conditions.

4. **Operational Transients:** RSSIM simulations help in defining procedures, setpoints, and providing an overall overview of reactor operation.





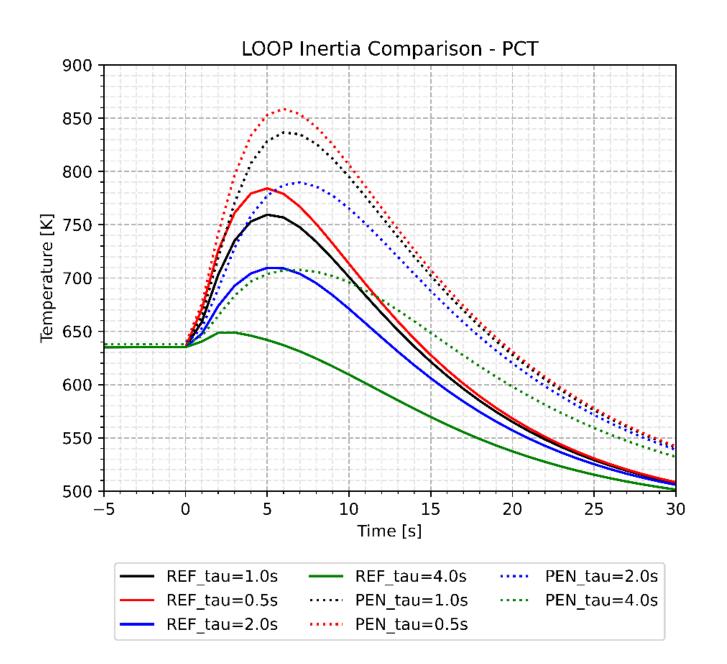
Pump Inertia Requirements

Loss of Off-Site Power



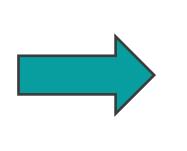
3. Pump Inertia Requirements: Loss of Off-Site Power

Support the definition of the requirement about the primary pump inertia.



	SAFETY CRITERIA	DECOUPLING CRITERIA
DBC1	No open cladding failure	PCT < 700 °C
DBC2	No open cladding failure (except random failure)	700 °C < PCT < 750 °C less than 10'
		750 °C < PCT < 800 °C less than 3'
DBC3	No systematic cladding failure	DBC2 criteria – case by case exceptions
DBC4	Cladding failures accepted consistently with radiological releases criteria	PCT < 800 °C
	No systematic clad melting	
	Fuel assembly allowing cooling function	
DEC – A	No propagation of a fuel melt	TBD

Simulations at different primary pump coastdown and with different penalizations.



Identification of the influence of each penalization, and investigation with different coastdown constants.



Unprotected Transients

Unprotected Transient of Overpower



4. Unprotected Transient Overpower – Reactivity insertion

Initiator: significant reactivity insertion

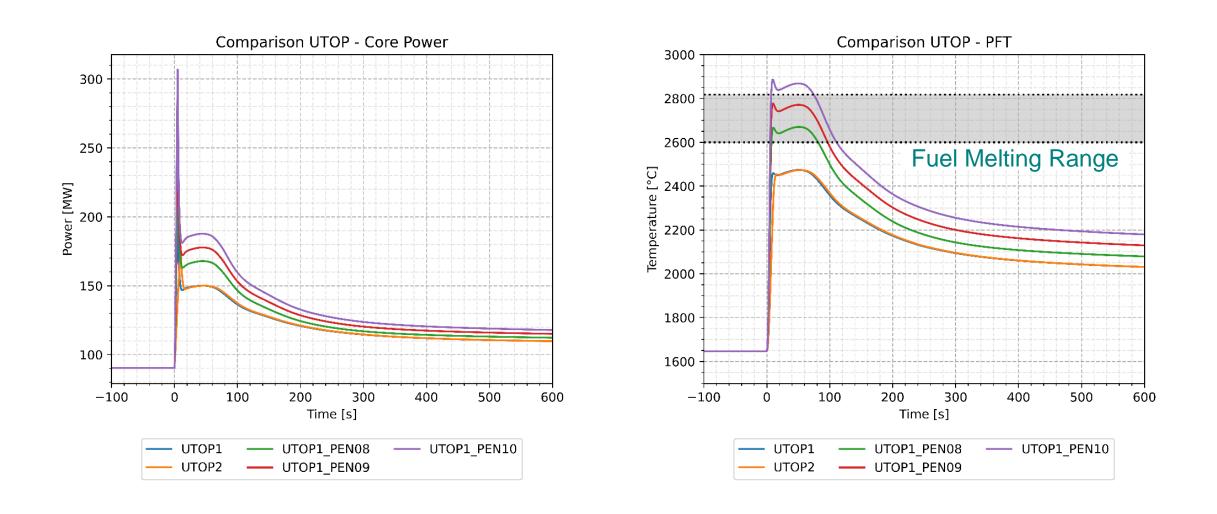
Failure of the shutdown function is postulated

Sensitivity studies to determine the maximum allowable reactivity insertion not causing fuel melting. Penalizations on doppler reactivity coefficients

Long-terms conditions (1h) in the reference case (NV=nominal value):

- Core power \rightarrow 117% NV
- Maximum cladding temperature \rightarrow 720 °C
- Core outlet temperature \rightarrow 651 °C

Objectives:





• Demonstrate the favorable intrinsic safety behavior of the system in challenging accidental conditions \rightarrow Practical elimination of generalised core melt scenarios • Inform the severe accident studies in terms of core state and vessel state

Operational Transients

Steam generator start-up



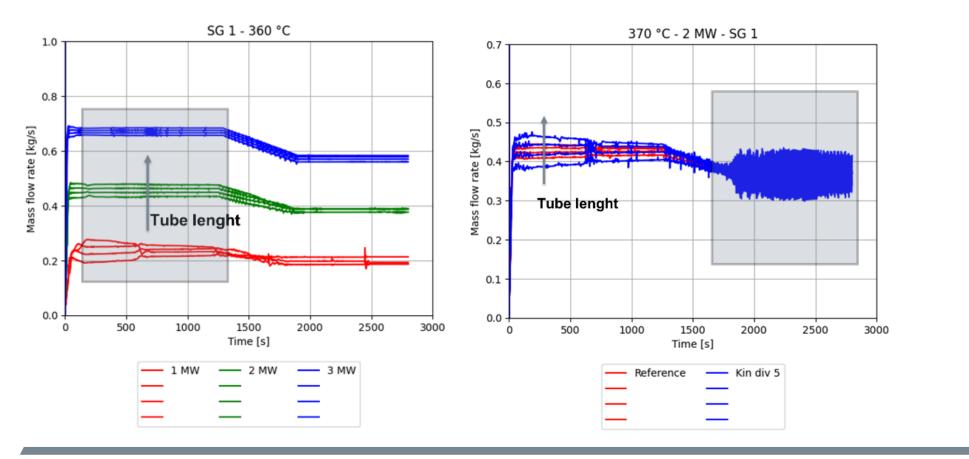
5. Operational Transient

Investigate a SG start-up procedure to minimise

- the water inventory in the SG tubes
- the risk of two-phase flow instabilities
- the risk of flow mal-distribution among the tubes,
- the thermal stress on reactor vessel, SG tubes, etc.

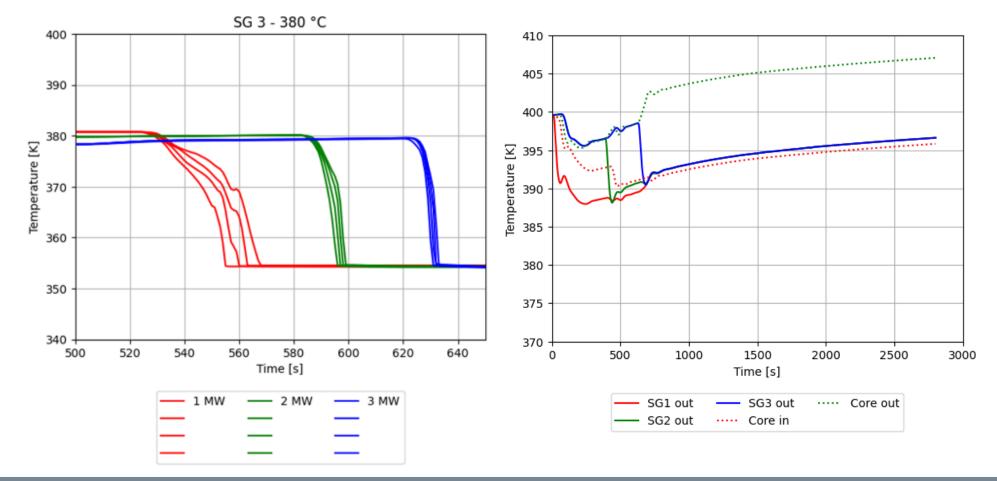
Challenges

• Impact of numerics, lack of closure laws, presence of non-condensable, lead freezing,



Address the onset of two-phase flow instabilities & SS flow maldistribution

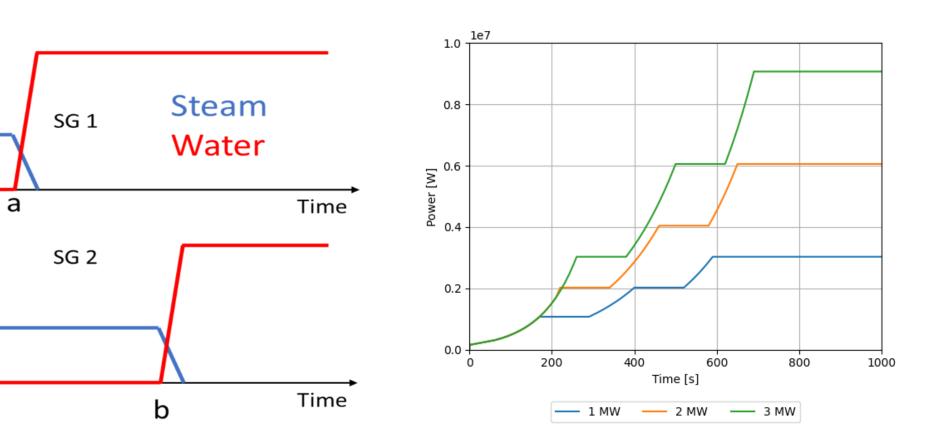




Preliminary procedure – Sequential SG startup

Flow rate

Flow rate



Address the thermal stress on metallic structures

CFD studies

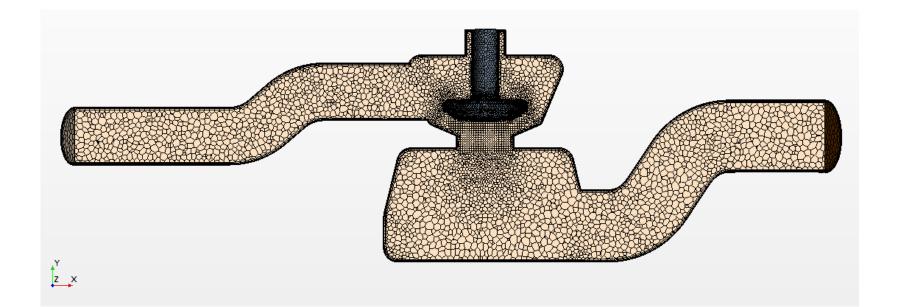


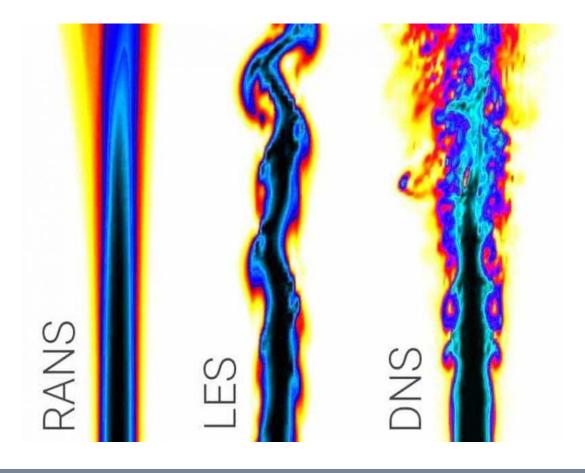


6.a CFD studies for plant thermal-hydraulics

- CFD = Computational Fluid Dynamics
 - Allow to solve the conservation equations of mass, momentum and energy on discretised geometries thanks to numerical alogorithms
 - Several approaches to deal with the turbulence
 - Reynolds Average Navier Stokes (average values and turbulence models),
 - Large Eddy Simulation (eddies larger than the mesh and models for smaller eddies),
 - Direct Numerical Simulation (all scales solved).
 - Steady state and transient analyses

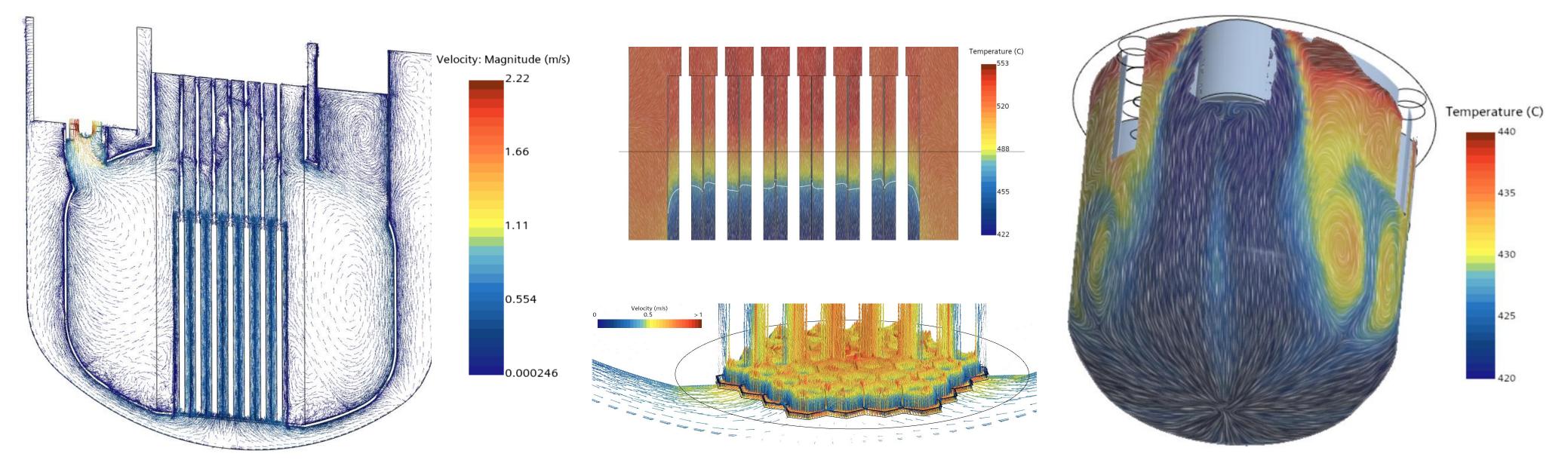






6.b CFD studies for plant thermal-hydraulics

Primary lead pool studies: to assess the coolant behavior in the reactor



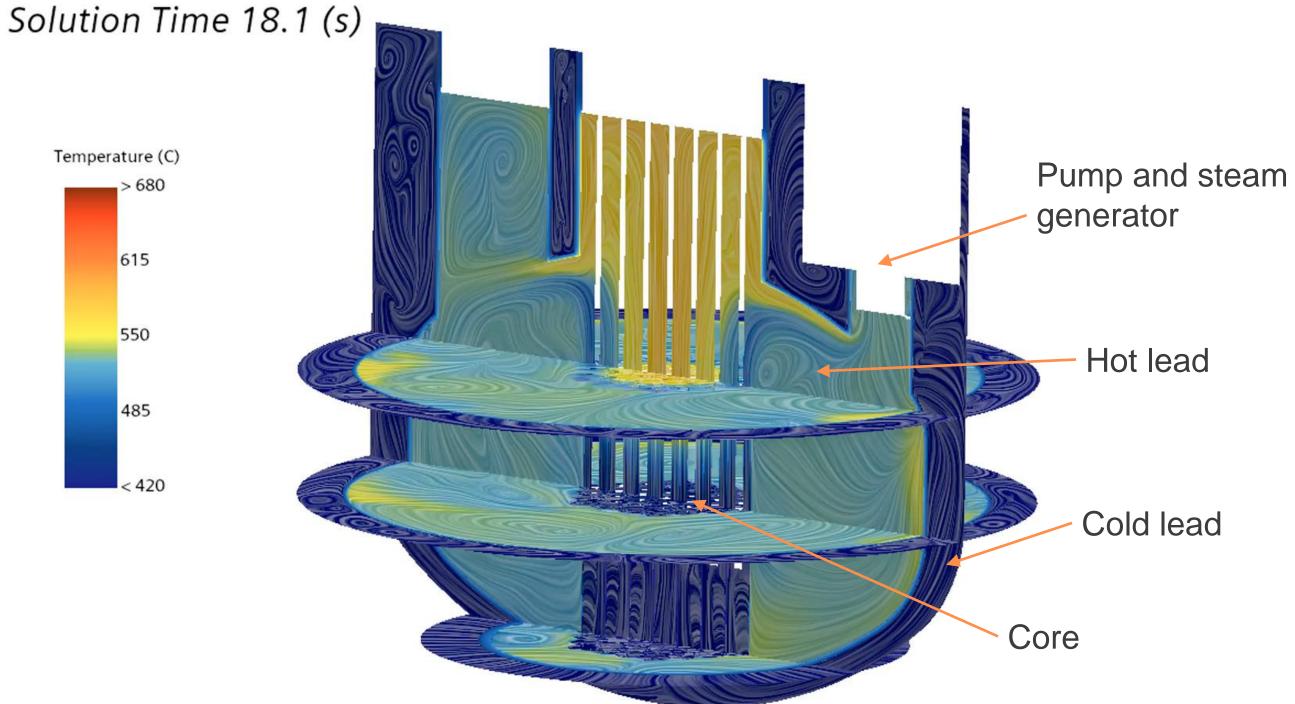
Mass flow and temperature distribution

 \rightarrow Allows improve the conception, to optimise the reactor operation and to assess the plant behaviour during normal operation or accidental conditions



6.C **CFD studies for plant thermal-hydraulics**

Example: loss of one primary pump transient (temperature and velocity streamlines) •

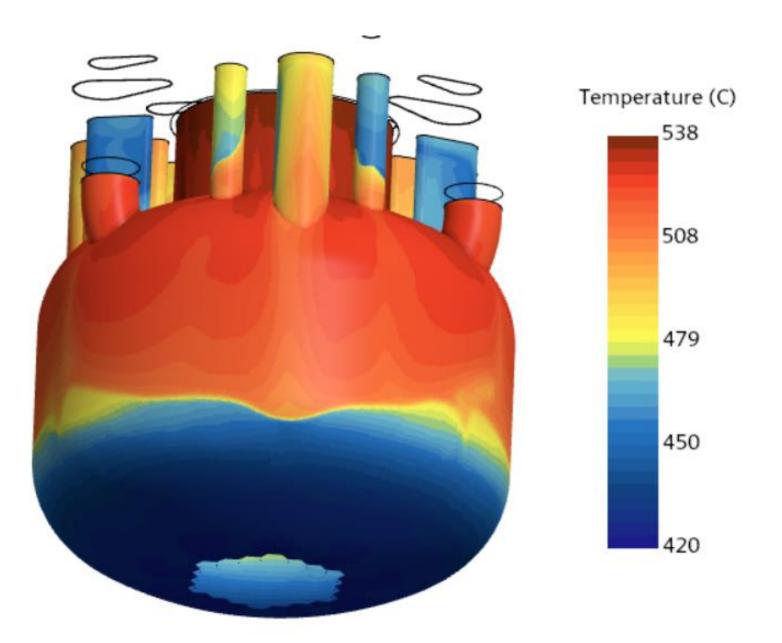






6.d CFD studies for plant thermal-hydraulics

Simulation for mechanical analyses: to provide thermal loading and assess the mechanical • constraints during the reactor operation



Temperature field on internal and external parts of the inner vessel

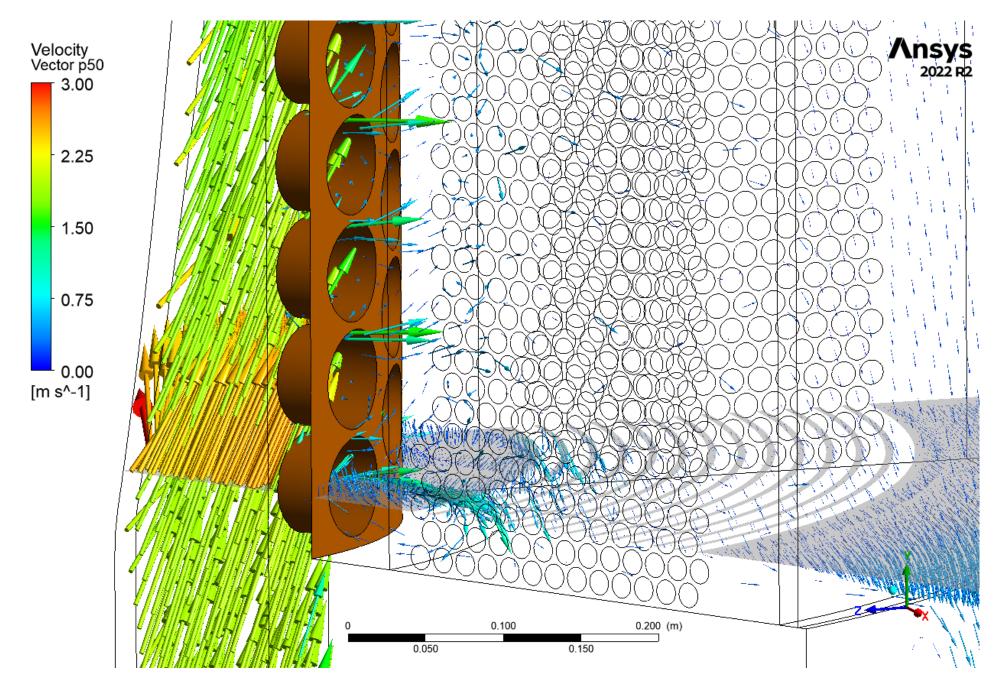






6.e CFD studies for plant thermal-hydraulics

Pump and heat exchangers analyses: to assess their performance, mechanical constraints, ...

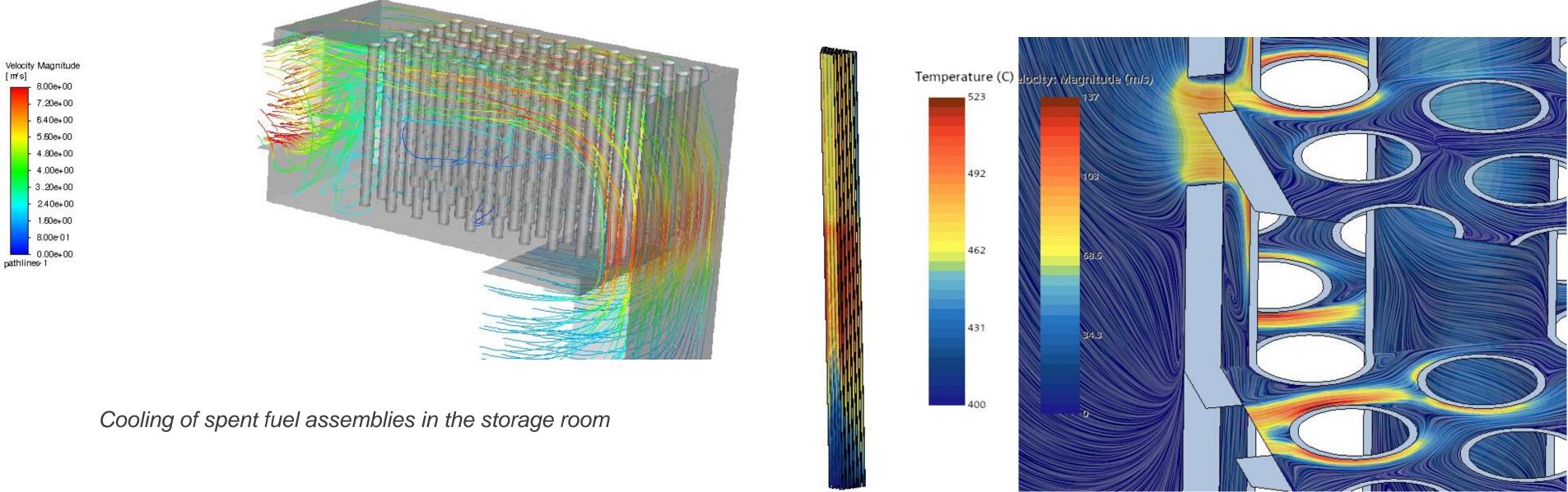


Velocity field at the steam generator inlet



6.f CFD studies for plant thermal-hydraulics

Fuel handling studies: to assess the thermal behavior of fresh and spent fuel assemblies for safety analyses









Temperature and gas flow inside a spent fuel assembly

CFD studies for plant thermal-hydraulics 6.g

- Other possible applications:
 - Coupling CFD / system code for better accuracy
 - Transport of species to assess the concentrations inside the primary coolant (radioactive) activation products, chemical species for corrosion risk,...)
 - Atmospheric dispersion (radiological emissions)
 - Artificial intelligence
 - . . .







Thank you

